LICENSE AMENDMENT

UTAH DEPARTMENT OF ENVIRONMENTAL QUALITY
DIVISION OF RADIATION CONTROL
RADIOACTIVE MATERIAL LICENSE

Pursuant to Utah Code Annotated, Title 19, Chapter 3 and the Radiation Control Rules, Utah Administrative Code (UAC) R313, and in reliance on statements and representations heretofore made by the Licensee designated below, a license is hereby issued authorizing the Licensee to transfer, receive, possess, and use the radioactive material designated below; and to use radioactive material for the purpose(s) and at the place(s) designated below. The license is subject to all applicable rules, and orders now or hereafter in effect and to all conditions specified below.

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LICENSEE

1. Name: Envirocare of Utah, Inc.

2. Address: 46 West Broadway
Suite 116
Salt Lake City, UT 84101

3. License Number UT 2300249
Amendment # 13

4. Expiration Date
October 22, 2003

5. License Category 4-a

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6. Radioactive Material (element and mass number)		7. Chemical and/or physical form		8. Average concentration per container on receipt	
A1.	Actinium-225	A1. through ABCDE inclusive. Notwithstanding Conditions 9 (Authorized Use), 16 (Prohibitions and Waste Requirements), and 56 (containerized waste), typically large volume , bulky or containerized, soil or debris. Debris can include both decommissioning (cleanup) and routinely generated operational waste including but not limited to radiologically contaminated paper, piping, rocks, glass, metal, concrete, wood, bricks, resins, sludge, tailings, slag, residues, personal protective equipment (PPE) that conforms to the size limitations in currently approved QA/QC Manual.	A1.	5.0e+02	pCi/g
B1.	Americium-241		B1.	1.0e+04	pCi/g
B2.	Americium-242		B2.	5.0e+02	pCi/g
B3.	Americium-242m		B3.	5.0e+02	pCi/g

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B4.	Americium-243		B4.	1.0e+04	pCi/g #
C1.	Antimony-126		C1.	5.0e+02	pCi/g
C2.	Antimony-126m		C2.	5.0e+02	pCi/g
C3.	Antimony-122		C3.	5.0e+02	pCi/g
C4.	Antimony-124		C4.	4.4e+08	pCi/g
C5.	Antimony-125		C5.	4.4e+08	pCi/g*
D1.	Arsenic-74		D1.	5.0e+02	pCi/g
D2.	Arsenic-73		D2.	5.0e+02	pCi/g
E1.	Barium-140		E1.	5.0e+02	pCi/g
E2.	Barium-133		E2.	1.0e+05	pCi/g
F1.	Berkelium-250		F1.	5.0e+02	pCi/g
F2.	Berkelium-249		F2.	5.0e+02	pCi/g
G.	Beryllium-7		G.	4.4e+08	pCi/g
H1.	Bismuth-206		H1.	5.0e+02	pCi/g
H2.	Bismuth-205		H2.	5.0e+02	pCi/g
H3.	Bismuth-207		H3.	5.0e+04	pCi/g
H4.	Bismuth-210m		H4.	5.0e+02	pCi/g
I1.	Cadmium-109		I1.	4.4e+08	pCi/g*
I2.	Cadmium-113m		I2.	5.0e+02	pCi/g
J1.	Calcium-47		J1.	5.0e+02	pCi/g
J2.	Calcium-45		J2.	4.4e+08	pCi/g
K1.	Californium-248		K1.	5.0e+02	pCi/g
K2.	Californium-250		K2.	5.0e+02	pCi/g
K3.	Californium-252		K3.	5.0e+02	pCi/g
L.	Carbon-14		L.	5.0e+05	pCi/g
M1.	Cerium-143		M1.	5.0e+02	pCi/g
M2.	Cerium-139		M2.	4.4e+08	pCi/g

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M3.	Cerium-141		M3.	4.4e+08	pCi/g
M4.	Cerium-144		M4.	4.4e+08	pCi/g*
N1.	Cesium-134		N1.	4.4e+08	pCi/g
N2.	Cesium-135		N2.	4.4e+08	pCi/g
N3.	Cesium-136		N3.	5.0e+02	pCi/g
N4.	Cesium-137		N4.	6.0e+04	pCi/g*
O.	Chromium-51		O.	4.4e+08	pCi/g
P1.	Cobalt-56		P1.	4.4e+08	pCi/g
P2.	Cobalt-57		P2.	4.4e+08	pCi/g
P3.	Cobalt-58		P3.	4.4e+08	pCi/g
P4.	Cobalt-60		P4.	3.0e+04	pCi/g
Q.	Copper-67		Q.	4.4e+08	pCi/g
R1.	Curium-241		R1.	5.0e+02	pCi/g
R2.	Curium-242		R2.	2.0e+06	pCi/g
R3.	Curium-243		R3.	1.0e+04	pCi/g
R4.	Curium-244		R4.	1.0e+04	pCi/g
R5.	Curium-245		R5.	5.0e+02	pCi/g
R6.	Curium-246		R6.	5.0e+02	pCi/g
R7.	Curium-247		R7.	5.0e+02	pCi/g
R8.	Curium-248		R8.	5.0e+02	pCi/g
S1.	Einsteinium-254		S1.	5.0e+02	pCi/g
S2.	Einsteinium-253		S2.	5.0e+02	pCi/g
T1.	Europium-152		T1.	2.0e+04	pCi/g
T2.	Europium-154		T2.	3.0e+04	pCi/g
T3.	Europium-155		T3.	4.4e+08	pCi/g
T4.	Europium-156		T4.	5.0e+02	pCi/g
U.	Fermium-252		U.	5.0e+02	pCi/g

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V1.	Gadolinium-148		V1.	5.0e+02	pCi/g
V2.	Gadolinium-153		V2.	4.4e+08	pCi/g
W.	Gallium-67		W.	5.0e+02	pCi/g
X.	Germanium-68		X.	4.4e+08	pCi/g*
Y1.	Gold-195		Y1.	4.4e+08	pCi/g
Y2.	Gold-198		Y2.	5.0e+02	pCi/g
Y3.	Gold-199		Y3.	5.0e+02	pCi/g
Z1.	Hafnium-175		Z1.	5.0e+02	pCi/g
Z2.	Hafnium-172		Z2.	5.0e+02	pCi/g
Z3.	Hafnium-181		Z3.	4.4e+08	pCi/g
AA.	Holmium-166m		AA.	5.0e+02	pCi/g
BB.	Hydrogen-3(Tritium)		BB.	2.5e+07	pCi/g
CC1.	Indium-114		CC1.	5.0e+02	pCi/g
CC2.	Indium-114m		CC2.	5.0e+02	pCi/g
CC3.	Indium-113m		CC3.	5.0e+02	pCi/g
CC4.	Indium-111		CC4.	5.0e+02	pCi/g
DD1.	Iodine-133		DD1.	5.0e+02	pCi/g
DD2.	Iodine-131		DD2.	5.0e+02	pCi/g
DD3.	Iodine-126		DD3.	5.0e+02	pCi/g
DD4.	Iodine-125		DD4.	4.4e+08	pCi/g
DD5.	Iodine-129		DD5.	3.1e+03	pCi/g
EE.	Iridium-192		EE.	4.4e+08	pCi/g
FF1.	Iron-52		FF1.	5.0e+02	pCi/g
FF2.	Iron-55		FF2.	4.4e+08	pCi/g
FF3.	Iron-59		FF3.	4.4e+08	pCi/g
FF4.	Iron-60		FF4.	5.0e+02	pCi/g
GG.	Krypton-85		GG.	5.0e+02	pCi/g

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HH.	Lanthanum-140		HH.	5.0e+02	pCi/g
II1.	Lead-203		II1.	5.0e+02	pCi/g
II2.	Lead-210		II2.	2.0e+06	pCi/g*
JJ1.	Manganese-52m		JJ1.	5.0e+02	pCi/g
JJ2.	Manganese-52		JJ2.	5.0e+02	pCi/g
JJ3.	Manganese-54		JJ3.	4.4e+08	pCi/g
KK1.	Mercury-194		KK1.	5.0e+02	pCi/g
KK2.	Mercury-203		KK2.	4.4e+08	pCi/g
LL.	Molybdenum-99		LL.	5.0e+02	pCi/g
MM1.	Neptunium-235		MM1.	5.0e+02	pCi/g
MM2.	Neptunium-237		MM2.	1.0e+04	pCi/g++
NN1.	Nickel-59		NN1.	1.4e+07	pCi/g
NN2.	Nickel-63		NN2.	2.2e+06	pCi/g
OO1.	Niobium-93m		OO1.	5.0e+02	pCi/g
OO2.	Niobium-94		OO2.	1.3e+04	pCi/g
PP1.	Osmium-191m		PP1.	5.0e+02	pCi/g
PP2.	Osmium-191		PP2.	5.0e+02	pCi/g
PP3.	Osmium-194		PP3.	5.0e+02	pCi/g
QQ.	Palladium-103		QQ.	5.0e+02	pCi/g
RR1.	Phosphorous-33		RR1.	5.0e+02	pCi/g
RR2.	Phosphorous-32		RR2.	5.0e+02	pCi/g
SS1.	Plutonium-236		SS1.	5.0e+02	pCi/g
SS2.	Plutonium-238		SS2.	1.0e+04	pCi/g
SS3.	Plutonium-239		SS3.	1.0e+04	pCi/g
SS4.	Plutonium-240		SS4.	1.0e+04	pCi/g
SS5.	Plutonium-241		SS5.	3.5e+05	pCi/g
SS6.	Plutonium-242		SS6.	1.0e+04	pCi/g

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SS7.	Plutonium-243		SS7.	5.0e+02	pCi/g
SS8.	Plutonium-244		SS8.	5.0e+02	pCi/g++
TT1.	Polonium-208		TT1.	5.0e+02	pCi/g
TT2.	Polonium-210		TT2.	4.4e+08	pCi/g
UU.	Potassium-40		UU.	1.0e+04	pCi/g
VV1.	Promethium-143		VV1.	5.0e+02	pCi/g
VV2.	Promethium-147		VV2.	4.4e+08	pCi/g*
WW1.	Radium-225		WW1.	5.0e+02	pCi/g
WW2.	Radium-226		WW2.	1.0e+04	pCi/g*
WW3.	Radium-228		WW3.	1.0e+04	pCi/g *
XX1.	Rhenium-188		XX1.	5.0e+02	pCi/g
XX2.	Rhenium-186		XX2.	5.0e+02	pCi/g
XX3.	Rhenium-184		XX3.	5.0e+02	pCi/g
XX4.	Rhenium-184m		XX4.	5.0e+02	pCi/g
XX5.	Rhenium-183		XX5.	5.0e+02	pCi/g
YY.	Rhodium-103m		YY.	5.0e+02	pCi/g
ZZ1.	Rubidium-86		ZZ1.	5.0e+02	pCi/g
ZZ2.	Rubidium-84		ZZ2.	5.0e+02	pCi/g
ZZ3.	Rubidium-82		ZZ3.	5.0e+02	pCi/g
ZZ4.	Rubidium-83		ZZ4.	4.4e+08	pCi/g*
AAA1.	Ruthenium-103		AAA1.	5.0e+02	pCi/g
AAA2.	Ruthenium-106		AAA2.	4.4e+08	pCi/g*
BBB1.	Samarium-153		BBB1.	5.0e+02	pCi/g
BBB2.	Samarium-145		BBB2.	5.0e+02	pCi/g
BBB3.	Samarium-151		BBB3.	4.0e+06	pCi/g
CCC1.	Scandium-47		CCC1.	5.0e+02	pCi/g
CCC2.	Scandium-44		CCC2.	5.0e+02	pCi/g

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CCC3.	Scandium-46		CCC3.	4.4e+08	pCi/g
DDD1.	Selenium-75		DDD1.	4.4e+08	pCi/g
DDD2.	Selenium-79		DDD2.	5.0e+02	pCi/g
EEE.	Silicon-32		EEE.	5.0e+02	pCi/g
FFF1.	Silver-111		FFF1.	5.0e+02	pCi/g
FFF2.	Silver-105		FFF2.	5.0e+02	pCi/g
FFF3.	Silver-108m		FFF3.	5.0e+04	pCi/g *
FFF4.	Silver-110m		FFF4.	4.4e+08	pCi/g *
GGG.	Sodium-22		GGG.	4.4e+08	pCi/g
HHH1.	Strontium-82		HHH1.	5.0e+02	pCi/g
HHH2.	Strontium-85		HHH2.	4.4e+08	pCi/g
HHH3.	Strontium-89		HHH3.	4.4e+08	pCi/g
HHH4.	Strontium-90		HHH4.	2.5e+04	pCi/g*
III.	Sulfur-35		III.	4.4e+08	pCi/g
JJJ.	Tantalum- 182		JJJ.	4.4e+08	pCi/g
KKK1.	Technetium-99m		KKK1.	5.0e+02	pCi/g
KKK2.	Technetium-95m		KKK2.	5.0e+02	pCi/g
KKK3.	Technetium-95		KKK3.	5.0e+02	pCi/g
KKK4.	Technetium-99		KKK4.	1.9e+05	pCi/g
LLL1.	Tellurium-125m		LLL1.	5.0e+02	pCi/g
LLL2.	Tellurium-123m		LLL2.	5.0e+02	pCi/g
MMM1.	Tellurium-129		MMM1.	5.0e+02	pCi/g
MMM2.	Tellurium-129m		MMM2.	5.0e+02	pCi/g
NNN.	Terbium-160		NNN.	5.0e+02	pCi/g
OOO1.	Thallium-202		OOO1.	5.0e+02	pCi/g
OOO2.	Thallium-201		OOO2.	5.0e+02	pCi/g
OOO3.	Thallium- 204		OOO3.	4.4e+08	pCi/g

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PPP1.	Thorium-231		PPP1.	5.0e+02	pCi/g
PPP2.	Thorium-229		PPP2.	5.0e+02	pCi/g
PPP3.	Thorium-230		PPP3.	1.5e+05	pCi/g
PPP4.	Thorium-232		PPP4.	1.0e+04	pCi/g*
QQQ1.	Thulium-170		QQQ1.	5.0e+02	pCi/g
QQQ2.	Thulium-171		QQQ2.	5.0e+02	pCi/g
RRR1.	Tin-121		RRR1.	5.0e+02	pCi/g
RRR2.	Tin-119m		RRR2.	5.0e+02	pCi/g
RRR3.	Tin-117m		RRR3.	5.0e+02	pCi/g
RRR4.	Tin-113		RRR4.	4.4e+08	pCi/g*
RRR5.	Tin-121m		RRR5.	5.0e+02	pCi/g
RRR6.	Tin-126		RRR6.	5.0e+02	pCi/g
SSS.	Titanium-44		SSS.	5.0e+02	pCi/g
TTT1.	Tungsten-188		TTT1.	5.0e+02	pCi/g
TTT2.	Tungsten-185		TTT2.	5.0e+02	pCi/g
TTT3.	Tungsten-181		TTT3.	5.0e+02	pCi/g
UUU1.	Uranium-232		UUU1.	7.5e+04	pCi/g
UUU2.	Uranium-233		UUU2.	7.5e+04	pCi/g
UUU3.	Uranium-234		UUU3.	3.7e+05	pCi/g
UUU4.	Uranium-235		UUU4.	1.9e+03	pCi/g*
UUU5.	Uranium-236		UUU5.	3.8e+05	pCi/g
UUU6.	Uranium-238		UUU6.	3.3e+05	pCi/g++
UUU7.	Uranium-depleted		UUU7.	3.7e+05	pCi/g++
UUU8.	Uranium-natural		UUU8.	6.8e+05	pCi/g++
VVV.	Vanadium-48		VVV.	5.0e+02	pCi/g
WWW1.	Xenon-133m		WWW1.	5.0e+02	pCi/g
WWW2.	Xenon-133		WWW2.	5.0e+02	pCi/g

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WWW3.	Xenon-131m		WWW3.	5.0e+02	pCi/g
WWW4.	Xenon-127		WWW4.	5.0e+02	pCi/g
XXX.	Ytterbium-169		XXX.	5.0e+02	pCi/g
YYY1.	Yttrium-88		YYY1.	4.4e+08	pCi/g
YYY2.	Yttrium-91		YYY2.	4.4e+08	pCi/g
ZZZ.	Zinc-65		ZZZ.	4.4e+08	pCi/g
AAAA1	Zirconium-88		AAAA1	5.0e+02	pCi/g
AAAA2	Zirconium-93		AAAA2	5.0e+02	pCi/g
AAAA3	Zirconium-95		AAAA3	4.4e+08	pCi/g*
ABCDE	Any Radioactive Material		ABCDE	2.0e+04	Curies***

Decay product Neptunium-239 is assumed to be present in concentration equal to Amercium-243.
* Decay products are assumed to be present in concentrations equal to the parent.
++ Short-lived decay products of U-238 (Th-234, Pa-234m and Pa-234); Np-237 (Pa-233); and Plutonium-244 (U-240, Np-240m, and Np-240) are assumed to be present in concentrations equal to the parent.
***Applies to undisposed maximum quantity at the Class A disposal cell.
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AUTHORIZED USE

9. A. Licensee may receive, store, and dispose by land burial, radioactive material as naturally occurring and accelerator produced material (NARM) and low-level radioactive waste. Prior to receiving an initial, low-level radioactive waste shipment for disposal from a generator, the Licensee shall obtain documentation which demonstrates that the low-level radioactive wastes have been approved for export to the Licensee. Approval is required from the low-level radioactive waste compact of origin (including the Northwest Compact), or for states unaffiliated with a low-level radioactive waste compact, the state of origin, to the extent a state can exercise such approval.

B. In accordance with Utah Code Annotated 19-3-105, the Licensee may not receive Class B or Class C low-level radioactive waste without first receiving approval from the Executive Secretary of the Utah Radiation Control Board and also receiving approval from the Governor and the Legislature.

C. The Licensee shall fulfill and maintain compliance with all conditions and shall meet all compliance

schedules stipulated in the Ground Water Discharge Permit, number UGW 450005, issued by the Executive Secretary of the Utah Water Quality Board

D. Notwithstanding Conditions 6 and 8, the Licensee, with prior written approval from the Executive Secretary on a case-by-case basis, may accept radionuclides additional to those listed in Condition 6 if the concentration of the unlisted radionuclide is less than or equal to 500 pCi/g and in the waste form specified by Condition 7.

E. Notwithstanding License Condition 6A1 through 6AAAA3 and 8A1 through 8AAAA3, the Licensee may dispose of Class A Low-Level Radioactive Waste and NARM in the Class A disposal cell described in License Condition 10.E. Class A waste is defined in Utah Radiation Control Rule R313-15-1008 and NARM at R313-12-3.

F. Effective January 1, 2002, the licensee shall not accept, possess, store or dispose of any radioactive waste delivered to the disposal site by any conveyance, unless the associated Shipping Documents have a valid Generator Site Access Permit number, issued by the Utah Division of Radiation Control, affixed.

SITE LOCATION

10. A. The Licensee may receive, store and dispose of licensed material at the Licensee's facility located in Section 32 of Township 1 South and Range 11 West, Tooele County, Utah.

B. Section 32, Township 1 South and Range 11 West, Tooele County, Utah, is defined by the following points of reference:

Southwest Section Corner:	Latitude 40 40' 51.894060" N Longitude 113 7' 28.579640" W
Elevation	4269.76 feet above mean sea level (amsl)
Southeast Section Corner	Latitude 40 41' 50.906471" N Longitude 113 6' 20.023247" W
Elevation	4277.27 feet-amsl
Northwest Section Corner	Latitude 40 41' 44.093832" N Longitude 113 7' 27.371551" W
Elevation	4273.06 feet-amsl
Northeast Section Corner	Latitude 40 40' 43.107203" N Longitude 113 6' 18.839771" W
Elevation	4280.83 feet-amsl

C. The Southwest Section Corner marker of Section 32 shall be the Point of Beginning (POB).

D. The Licensee shall cause a survey to be conducted by a Utah licensed land surveyor to identify the section corners of Section 32, Township 1 South, and Range 11 West, Tooele County, Utah (as defined in Condition 10.B). Licensee shall place monuments with brass caps at the identified section corner locations. Monuments shall be permanent and constructed in a manner that will protect them from being disturbed.

E. The Class A disposal cell shall be defined by the area enclosed by the following points of reference:

Location	Latitude	Longitude
NW corner	40 41' 28.004487" N	113 7' 23.847971" W
SW corner	40 41' 14.175042" N	113 7' 24.153414" W
SE corner	40 41' 13.717662" N	113 6' 54.827468" W
NE corner	40 41' 27.547403" N	113 6' 54.521700" W

F. The Containerized Waste Facility within the Class A disposal cell will be separated from the non-containerized area by a 6-foot chain link fence on the berm around the Containerized Waste Facility perimeter area.

11. The open cell area within the LARW and Class A disposal embankments where waste disposal/placement has or may occur, but the cover system has not been completed shall be limited to 1,950,000 square feet. Uncovered radioactive waste shall be limited to a surface area of 1,020,000 square feet.

12. Pursuant to UAC R313-12-55(1), the Licensee is granted an exemption to UAC R313-25-9, as it relates to land ownership and assumption of ownership.

SPECIAL NUCLEAR MATERIAL

13. In accordance with the Order issued by the U.S. Nuclear Regulatory Commission dated May 24, 1999, Docket No. 040-8989, License No. SMC-1559, the Licensee may possess Special Nuclear Material (SNM) within the restricted area of the Licensee's facility as described in Condition 10 provided that:

A. Concentrations of SNM in individual waste containers must not exceed the values Listed in Table 13-A at time of receipt:

Column 1 Radionuclide	Column 2 Maximum Concentration (pCi/g)	Column 3 Measurement Uncertainty (pCi/g)
U-235[a]	1,900	285
U-235[b]	1,190	179
U-235[c]	160	24
U-235[d]	680	102
U-233	75,000	11,250
Pu-236	500	75
Pu-238	10,000	1,500
Pu-239	10,000	1,500
Pu-240	10,000	1,500
Pu-241	350,000	50,000
Pu-242	10,000	1,500
Pu-243	500	75
Pu-244	500	75

a - for uranium below 10 percent enrichment and a maximum of 20 percent MgO of the weight of the waste
b - for uranium at or above 10 percent enrichment and a maximum of 20 percent MgO of the weight of the waste
c - for uranium at any enrichment with unlimited MgO or beryllium
d - for uranium at any enrichment with sum of MgO and beryllium not exceeding 49 percent of the weight of the waste

*The measurement uncertainty values in Column 3 above represent the maximum one-sigma uncertainty associated with the measurement of the concentration of the particular radionuclide.

The SNM must be homogeneously distributed throughout the waste. If the SNM is not homogeneously distributed, then the limiting concentrations must not be exceeded on average in any contiguous mass of 145 kilograms.

B. Except as allowed by notes a, b, c, and d in Condition 13A, waste must not contain “pure forms” of

chemicals containing carbon, fluorine, magnesium, or bismuth in bulk quantities (e.g., a pallet of drums, a B-25 box). By "pure forms," it is meant that mixtures of the above elements such as magnesium oxide, magnesium carbonate, magnesium fluoride, bismuth oxide, etc. do not contain other elements. These chemicals would be added to the waste stream during processing, such as at fuel facilities or treatment such as at mixed waste treatment facilities. The presence of the above materials will be determined by the generator, based on process knowledge or testing.

C. Except as allowed by notes c and d in Condition 13A, waste accepted must not contain total quantities of beryllium, hydrogenous material enriched in deuterium, or graphite above one percent of the total weight of the waste. The presence of the above materials will be determined by the generator, based on process knowledge, physical observations, or testing.

D. Waste packages must not contain highly water soluble forms of uranium greater than 350 grams of uranium-235 or 200 grams of uranium-233. The sum of the fractions rule will apply for mixtures of U-233 and U-235. Highly soluble forms of uranium include, but are not limited to: uranium sulfate, uranyl acetate, uranyl chloride, uranyl formate, uranyl fluoride, uranyl nitrate, uranyl potassium carbonate, and uranyl sulfate. The presence of the above materials will be determined by the generator, based on process knowledge or testing.

E. Mixed waste processing of waste containing SNM will be limited to stabilization (mixing waste with reagents), micro-encapsulation, and macro-encapsulation using low-density polyethylene.

F. Envirocare shall require generators to provide the following information for each waste steam:

Before Receipt

1. Waste Description. The description must detail how the waste was generated, list the physical forms in the waste, and identify uranium chemical composition.
2. Waste Characterization Summary. The data must include a general description of how the waste was characterized (including the volumetric extent of the waste, and the number, location, type, and results of any analytical testing), the range of SNM concentration ranges, and the analytical results with error values used to develop the concentration ranges.
3. Uniformity Description. A description of the process by which the waste was generated showing that the spatial distribution of SNM must be uniform, or other information supporting spatial distribution.
4. Manifest Concentration. The generator must describe the methods to be used to determine the concentrations on the manifests. These methods could include direct measurement and the use of scaling factors. The generator must describe the uncertainty associated with sampling and testing used to obtain the manifest concentrations.

   Envirocare shall review the above information and, if adequate, approve in writing this pre-shipment waste characterization and assurance plan before permitting the shipment of a waste stream. This will include statements that Envirocare has a written copy of all the information required above, that the characterization information is adequate and consistent with the waste description, and that the information is sufficient to demonstrate compliance with Conditions 13.F.1 through 13.F.4. Where generator process knowledge is used to

demonstrate compliance with Conditions 13.A, 13.B, 13.C, or 13.D, Envirocare shall review this information and determine when testing is required to provide additional information in assuring compliance with the conditions. Envirocare shall retain this information as required by the State of Utah to permit independent review.

At Receipt

Envirocare shall require generators of SNM waste to provide a written certification with each waste manifest that states the SNM concentrations reported on the manifest do not exceed the limits in Condition 13.A, that the measurement uncertainty does not exceed the uncertainty value in Condition 13.A, and that the waste meets Conditions 13.B through 13.D.

G. Sampling and radiological testing of waste containing SNM must be performed in accordance with the Utah Division of Radiation Control License Condition 58.

H. Envirocare shall notify the NRC, Region IV office within 24 hours if any of the above conditions are violated. A written notification of the event must be provided within 7 days.

I. Envirocare shall obtain NRC approval prior to changing any activities associated with the above conditions.

J. Notwithstanding License Condition 13.A through 13.I, for the Containerized Waste Facility described in License Condition 10.F, the following limits for possession of SNM apply only at the Containerized Waste Facility:

Consistent with the definition of special nuclear material given in UAC R313-12-3, the maximum quantity of special nuclear material which the Licensee may possess at any one time, shall not exceed: 350 grams of U-235, 200 grams of U-233, and 200 grams Pu, or any combination of them in accordance with the following formula:

$$\frac{(\text{Grams U-235})}{350} + \frac{(\text{Grams U-233})}{200} + \frac{(\text{Grams Pu})}{200} \quad 1$$

"Possession" and "Disposal" are defined in License Conditions 63 and 64 respectively.

MIXED WASTE

14. A. The Licensee may receive for treatment, storage, and disposal any radioactive waste as authorized by this license that is also determined to be hazardous (commonly referred to as mixed waste) as permitted by the "Hazardous Waste Plan Approvals" issued and modified by the Executive Secretary, Utah Solid and Hazardous Waste Control Board and "HSWA Permit" issued by the U.S. Environmental Protection Agency.

B. The Licensee shall dispose of these wastes in the "mixed waste" disposal embankment only.

Characteristic or listed hazardous waste treated at the Envirocare facility shall not be disposed of in the Low Activity Radioactive Waste (LARW) embankment or the Class A disposal cell.

15. Reserved

PROHIBITIONS AND WASTE ACCEPTANCE REQUIREMENTS

16. A. Sealed sources as defined in Utah Administrative Code (UAC)R313-12 shall not be accepted for disposal.

B. In accordance with UAC R313-15-1008(2)(a)(v), waste shall not be readily capable of detonation or of explosive decomposition or reaction at normal pressures and temperatures, or of explosive reaction with water.

C. In accordance with UAC R313-15-1008(2)(a)(vi), waste shall not contain, or be capable of generating, quantities of toxic gases, vapors, or fumes harmful to persons transporting, handling, or disposing of the waste.

D. In accordance with UAC R313-15-1008(2)(a)(vii), waste shall not be pyrophoric.

E. Waste containing untreated biological, pathogenic, or infectious material including radiologically contaminated laboratory research animals is prohibited

F. Receipt of liquid radioactive waste is prohibited.

G. In accordance with UAC R313-15-1008(2)(a)(viii), gaseous waste received for disposal in the Containerized Waste Facility shall be packaged at an absolute pressure that does not exceed 1.5 atmospheres at a temperature of 20 degrees Celsius and the total activity of any container shall not exceed 100 curies (3.7×10^{12} Bequerels).

H. In accordance with UAC R313-15-1008(2)(a)(ii), waste received for disposal in the Containerized Waste Facility shall not be packaged in cardboard or fiberboard containers.

I. The Licensee shall not accept for disposal any neutron source (e.g., polonium-210, americium-241, radium-226 in combination with beryllium or other target).

J. Incinerator ash shall be solidified, granular, treated, or encapsulated in controlled low-strength material (CLSM) in a manner that renders it non-dispersible in air.

K. Radioactive waste containing chelating agents greater than 0.1 percent by weight shall be disposed of in the Mixed Waste Cell.

L. The Licensee shall not accept containerized radioactive waste unless each waste package has been:

i. Classified in accordance with R313-15-1008, "Classification and Characteristics of Low-Level Radioactive Waste." In addition, the Licensee shall require that all radioactive waste received for disposal meet the requirements specified in the Nuclear Regulatory Commission, " Branch Technical Position on Concentration Averaging and Encapsulation", as amended.
ii. Marked as either Class A Stable or Class A Unstable as defined in the most recent version of the "Low-Level Waste Licensing Branch Technical Position on Radioactive Waste Classification." originally issued May, 1993 by the U.S. Nuclear Regulatory Commission.
iii. Marked with a unique package identification number, clearly visible on the package, that can be correlated with the manifest for the waste shipment in which the package arrives at the facility.

M. The Licensee may accept containerized Class A LLRW in the following waste packages for disposal in the Containerized Waste Facility of the Class A disposal cell:

i DOT "strong, tight" containers in accordance with 49 CFR 173 and meeting the following void space criteria: void spaces within the waste and between the waste and its packaging shall be reduced to the extent practicable, but in no case shall less than 85 percent of the capacity of the container be filled.
ii. High-Integrity Containers (HICs) exceeding the void space criteria provided in License Condition 16.M.i, shall be approved by the Executive Secretary.
iii DOT "strong, tight" containers in accordance with 49 CFR 173 exceeding the void space criteria provided in License Condition 16.M.i and large components shall be placed as approved by the Executive Secretary.
iv Oversized DOT containers (larger than 215 cubic feet) meeting the void space criteria provided in License Condition 16.M.i shall placed in accordance with the current approved Construction QA/QC Manual.

MANAGEMENT OF FREE LIQUIDS

17. In accordance with UAC R313-15-1008(2)(a)(iv), solid waste received for disposal shall contain as little free-standing and non-corrosive liquid as reasonably achievable, but shall contain no more free liquids than one percent of the volume of the waste. Radioactive waste containing free liquid shall be managed in accordance with the LLRW Waste Management Plan identified at License Condition 59.

RADIATION SAFETY

18. The Licensee shall comply with the provisions of UAC R313-18, "Notices, Instructions and Reports to Workers by License or Registrants, Inspections" and UAC R313-15, "Standards for Protection Against Radiation."

19. The Licensee may transport licensed material or deliver licensed material to a carrier for transport in

accordance with the provisions of UAC R313-19-100, "Transportation."

20. Written procedures incorporating operating instructions and appropriate safety precautions for licensed activities shall be maintained and available at the location specified in License Condition 10.A. The written procedures established shall include the activities of the radiation safety and environmental monitoring programs, the employee training program, operational procedures, analytical procedures, and instrument calibration. At least annually, the Licensee shall review all procedures to determine their continued applicability.

21. A. The Licensee's Bulk Waste Facility Radiation Protection Manager shall review and approve written procedures as stated in License Condition 20 and subsequent changes to the procedures related to bulk waste disposal operations.

B. The Licensee's Containerized Waste Facility Radiation Protection Manager shall review and approve written procedures as stated in License Condition 20 and subsequent changes to the procedures related to containerized waste disposal operations.

ROUTINE MONITORING AND CONTAMINATION SURVEYS

22. The Licensee shall conduct contamination surveys in accordance with Table 22-A:

TABLE 22-A.

Type	Location	Frequency
A. Gamma Radiation Levels	1. Perimeter of Restricted Area(s)	1. Weekly
	2. Office Area (s)	2. Weekly
	3. Lunch/Change Area(s)	3. Weekly
	4. Transport Vehicles	4. Upon vehicle arrival at site and before departure.
	5. Box counter facility	6. Weekly
	6. Decontamination facilities	6. Weekly
B. Contamination Wipes	1. Eating Area(s) 2. Change Area(s)	1. Weekly 2. Weekly
	3. Office Areas(s)	3. Weekly
	4. Railcar rollover and control shack	4. Weekly
	5. Equipment/Vehicles	5. Once before release
	6. Decontamination facilities	6. Weekly
C. Employee/Personnel	1. Skin & Personal clothing	1. Prior to exiting controlled area
D. Gamma Exposure	1. Administration Bldg.(s)	1. Quarterly
E. Radon Concentration	1. Administration Bldg.(s)	1. Quarterly

23. The Licensee shall determine internal exposure of employees under its bioassay program, in accordance

with UAC R313-15-204.

24. The Licensee shall implement a respiratory protection program that is in accordance with UAC R313-15-703.

25. The Licensee shall calibrate air sampling equipment at intervals not to exceed six months.

26. The operational environmental monitoring program shall be conducted in accordance with the License Renewal Application, Appendix R (revised), dated August 15, 2001.

27. Vehicles, containers, facilities, materials, equipment or other items for unrestricted use, except conveyances used for commercial transport of radioactive waste, shall not be released from the Licensee's control if contamination exceeds the limits found in Table 27-A.

TABLE 27-A

Nuclide[a]	Column 1 Average [b,c,f]	Column 2 Maximum [b,d,f]	Column 3 Removable [b,e,f]
U-nat, U-235, U-238, and associated decay products	5,000 dpm alpha/ 100cm^2	15,000 dpm alpha/ 100cm^2	1,000 dpm alpha/ 100cm^2
Transuranics, Ra-226, Ra-228, Th-230, Th-228, Pa-231, Ac-227, I-125, I-129	100 dpm/100cm^2	300 dpm/100cm^2	20 dpm/100cm^2
Th-nat, Th-232, Sr-90, Ra-223, Ra-224, U-232, I-126, I-131, I-133	1,000 dpm/100cm^2	3,000 dpm/100cm^2	200 dpm/100cm^2
Beta-gamma emitters (nuclides with decay modes other than alpha emissions or spontaneous fission) except Sr-90 and other noted above.	5,000 dpm beta, gamma/100cm^2	15,000 dpm beta-gamma/100cm^2	1,000 dpm beta-gamma/100cm^2

a. Where surface contamination on both alpha-and beta-gamma emitting nuclides exists, the limits established for alpha-and beta-gamma emitting nuclides should apply independently.
b. As used in this table, dpm (disintegration's per minute) means the rate of emission by radioactive material as determined by correcting the counts per minute observed by an appropriate detector for background, efficiency, and geometric factors associated with the instrumentation.
c. Measurements of average contamination should not be averaged over more than one square meter. For objects of less surface area, the average should be derived for each such object.
d. The maximum contamination level applies to an area of not more than 100 cm^2.
e. The amount of removable radioactive material per 100 cm^2 of surface area should be determined by wiping the area with dry filter or soft absorbent paper, applying moderate pressure, and assessing the amount of radioactive material on the wipe with an appropriate instrument of known efficiency.

When removable contamination on objects of less surface area is determined, the pertinent levels should be reduced proportionally and the entire surface should be wiped.

f. The average and maximum radiation levels associated with surface contamination resulting from beta-gamma emitters shall not exceed 0.2 mrad/hr at 1 cm and 1.0 mrad/hr at 1 cm, respectively, measured through not more than 7 milligrams per square centimeter of total absorber.

REPORTING

28. Reserved.

29. The Licensee shall submit the following reports to the Executive Secretary:

A. Quarterly results from the Environmental Monitoring Program (Appendix R, as amended). The report(s) shall be submitted within 90 days after the expiration of each calender quarter. Calender Quarter shall mean:

First Quarter	January, February, and March
Second Quarter	April, May, and June
Third Quarter	July, August, and September
Fourth Quarter	October, November, and December

B. An annual summary of the volume and tonnage, radioisotopes and their activities for materials disposed of by March 31st of each year for the preceding year.

C. A monthly disposal report of NARM waste, low-level radioactive waste, mixed waste, and uranium/thorium mill tailings (i.e., 11e. (2) wastes) to include the volume and tonnage in tons, cubic yards, and cubic feet. The monthly report shall be submitted within 30 days from the end of the preceding month.

D. Except for waste received and disposed of in the Containerized Waste Facility, the Licensee shall submit in writing on a quarterly basis to the Executive Secretary, a report that compares the actual radioactivity received and disposed of with the Weighted Average Concentrations provided in the License Renewal Application of March 16, 1998, Appendix P, Table 5. The report(s) shall be submitted within 30 days after the expiration of each calender quarter. Calender Quarter shall mean:

First Quarter	January, February, and March

Second Quarter	April, May, and June
Third Quarter	July, August, and September
Fourth Quarter	October, November, and December

E. For the Class A disposal cell, the Licensee shall ensure that the maximum acceptable activities used as source terms in the groundwater performance modeling are not exceeded after facility closure. Therefore the Licensee shall notify the Executive Secretary, at the earliest knowledge, that the following nuclides are scheduled for disposal: aluminum-26, berklium-247, calcium-41, californium-249, californium-250, chlorine-36, rhenium-187, terbium-157, and terbium-158.

F. An annual report shall be submitted by March 31st and shall report the cumulative void space (expressed as a percent of waste volume) disposed of in the Containerized Waste Facility for the previous year.

30. Except as provided by this condition, the Licensee shall maintain the results of sampling, analyses, surveys, and instrument calibration, reports on inspections and audits, employee training records as well as any related reviews, investigations and corrective actions, for five (5) years. The Licensee shall maintain personnel exposure records in accordance with UAC R313-15-201.

STAFFING/QUALIFICATIONS

31. A. Radiation Safety operations for bulk waste(s) shall be conducted by or under the supervision of Tye Rogers, Bulk Waste Facility Radiation Protection Manager.

B. Radiation Safety operations for containerized waste(s) shall be conducted by or under the supervision of Mark Ledoux, Containerized Waste Facility Radiation Protection Manager.

32. The Licensee's staff shall meet the qualifications as described in Appendix I (April 12, 2001, rev.13) of the License Renewal Application (dated March 16, 1998).

CONSTRUCTION ACTIVITIES

33. The Licensee shall obtain prior written approval from the Executive Secretary prior to construction of significant facilities. Significant facilities shall include, but are not limited to waste, stormwater, and wastewater related handling, storage, and transfer projects.

34. Reserved

35. Reserved
36. Reserved

37. All ion exchange resins shall be disposed of as follows:

    A. Solidified using solidification agents approved by the Executive Secretary and disposed of in the Containerized Waste Facility; or

    B. Packaged in High-Integrity Containers (HIC) approved by the Executive Secretary, carbon-steel liners, unapproved HICs, or poly HICs meeting the void space criteria described in License Condition 16.M.i and disposed of in the Containerized Waste Facility; or

    C. Packaged in High-Integrity Containers (HIC) approved by the Executive Secretary, carbon-steel liners, unapproved HICs, or poly HICs not meeting the void space criteria described in License Condition 16.M.i and disposed of as approved by the Division under License Condition 16.M.ii or 16.M.iii in the Containerized Waste Facility; or

    D. Disposed of in bulk waste placement areas of the LARW and Class A disposal cells by blending with clay in accordance with the requirements of the LLRW Waste Management Plan provided at License Condition 59.

38. The Licensee shall construct the Class A disposal cell identified in License Condition 10.E in accordance with approved engineering design drawings "Series 9821".

39. Waste placement and backfilling within the Containerized Waste Facility identified in License Condition 10.F shall be conducted in accordance with the following:

    A. The containerized waste embankment shall conform to the characteristics defined, analyzed, and described in the Engineering Justification Report “Class A Disposal Cell Containerized Waste Facility” (dated April 12, 2001); Engineering Justification Report, Addendum “Fifteen Percent Void Space Criteria” (Revision 1 dated October 10, 2001); and the AMEC letter to Envirocare of Utah, Inc. “Placement of Drums and B-25 Containers with 15 Percent Voids; Envirocare Class A - Containerized Waste Facility Near Clive, Utah” (dated October 2, 2001).

    B. Waste container placement configurations and associated waste placement procedures, backfill materials and procedures, and backfill cover materials shall be those approved by the Executive Secretary following testing according to Work Element: Containerized Waste Facility-Waste Placement Test Pad of the currently approved Construction Quality Assurance/Quality Control Manual.

40. In the LARW cell, disposal of mobile waste shall be restricted to the top slope area of the LARW cell, as limited by points of reference in Table 40-A. In the LARW Cell, waste lifts containing any mobile waste

shall be designated as mobile waste lifts and shall be disposed of within the mobile waste boundaries.

TABLE 40-A

Boundaries of Mobile Waste Disposal and LARW Cell Topslope Design	Points of Reference	
	Latitude	Longitude
LARW Cell Waste Disposal Boundaries		
Northeast Corner	40 41' 10.700524" N	113 6' 36.372920" W
Southeast Corner	40 40' 52.230624" N	113 6' 36.713462" W
Southwest Corner	40 40' 52.379041" N	113 6' 51.184491" W
Northwest Corner	40 41' 10.851418" N	113 6' 50.846182" W
Mobile Waste Disposal Boundaries		
Northeast Corner	40 41' 10.206265" N	113 6' 37.022208" W
Southeast Corner	40 40' 52.823735" N	113 6' 37.391607" W
Southwest Corner	40 40' 52.972152" N	113 6' 50.405346" W
Northwest Corner	40 41' 10.357159" N	113 6' 50.196894" W
LARW Topslope Cover Design Boundaries (Envirocare Drawing 9407-4, Rev. I, 2/18/98		
Northeast Corner	40 41' 7.381332" N	113 6' 38.365806" W
Southeast Corner	40 41' 55.771282" N	113 6' 38.838893" W
Southwest Corner	40 40' 55.722242" N	113 6' 53.177077" W
Northwest Corner	40 41' 7.285150" N	113 6' 52.874199" W

41. The LARW Cell topslope shall be constructed in accordance with all engineering design and specifications approved by this license, and as restricted to the points of reference, provided in Table 40-A.

42. "Mobile wastes" are defined as any waste containing any quantity of the following isotopes: carbon-14, iodine-129, neptunium-237, sodium-22, technetium-99, hydrogen-3 (tritium), amercium-242m, bismuth-210m, curium-245, curium-246, curium-248, gadolinium-148, iron-60, mercury-194, holmium-166m, selenium-79, silicon-32, tin-121m, tin-126, thorium-229, titanium-44, and zirconium-93.

43. Reserved

44. The Licensee shall fulfill all requirements and maintain compliance with all conditions in the CQA/QC Manual and Engineering Drawings currently approved by the Executive Secretary.

45. Reserved

46. Reserved

47. The Licensee shall not initiate disposal operations in newly excavated areas until the Division has inspected and the Executive Secretary has approved the cell/embankment liner.

CONSTRUCTION DRAWINGS.

48. A. The Licensee shall provide a comprehensive set of drawings for the entire Clive site. The drawings shall correctly: (1) locate all structures, utilities, fences, ponds, drainage features railroad tracks, roads, storage facilities, loading and off-loading facilities, disposal embankments, all environmental monitoring locations including instruments/devices, and any other appurtenances related to the operation, maintenance and closure of the disposal facility; and (2) provide structural details including site elevation. A directory shall be included that identifies drawings by discrete number, title, date and revision. The drawings shall indicate as-built conditions as they existed no earlier than 30 days prior to the submittal. Drawings of finished construction shall be marked as "As-Built."

B. Drawings showing approved future designs, shall be marked as "Record Drawings." Record drawings or construction drawings shall be certified by a Utah registered professional engineer.

C. Within 30 days of the completion of any project that requires approval by the Executive Secretary, a set of "As-Built" drawings shall be submitted for review and inclusion into the comprehensive drawing set.

SITE OPERATING PROCEDURES

49. Reserved

50. Reserved

51. Reserved

52. Reserved

53. Reserved

54. Reserved

55. For the Class A disposal cell, the Licensee shall ensure that the actual cumulative activity of chlorine-36 does not exceed 0.2828 picocuries per gram in accordance with the following formula:

$$\frac{\text{Total Activity of chlorine-36 Received (picocuries)}}{\text{Total Mass of Active Cell (grams) + Completed Cell(grams)}} \quad 0.2828 \text{ picocuries per gram}$$

56. Containerized Class A waste shall be certified by the generator to meet the Licensee's Waste Acceptance Criteria in accordance with the Waste Characterization Plan described in License Condition 58.

57. The Licensee may perform the following activities on rail lines, not including the main line. adjacent to Section 32:

A. Visual inspections
B. Radiation level surveys
C. Affix labels

58 The Licensee shall fulfill and maintain compliance with all conditions and requirements in the LLRW Waste Characterization Plan (dated April 12, 2001).

59. The Licensee shall fulfill and maintain compliance with all conditions and requirements in the LLRW Waste Management Plan (dated April 12, 2001).

60. All wind dispersed litter, located outside of the disposal cell/embankments, shall be retrieved by the Licensee and returned to the Licensee's control within 24 hours

61. Truck, railcar, and other equipment washdown (decontamination) facilities, including evaporation ponds, shall be controlled with fences or other approved barriers to prevent intrusion.

62. All burial embankments and waste storage areas, including immediately adjacent drainage structures, shall be controlled areas, surrounded by a six-foot high chain link fence. All permanent fences shall be six-feet high chain link topped with three strand barbed wire, top tension wire, and twisted selvedge.

63. Radioactive and mixed waste within Section 32 and all rail spurs controlled by the Licensee around the Licensee's Disposal Facility is possessed by the Licensee. Waste conveyed to the facility by truck is in transport as long as the commercial carrier driver and vehicle remain at the Clive disposal facility. The Licensee does not possess such waste for purposes of determining compliance with surety requirements and SNM quantity limits, except that the Licensee does, however, possess any waste containing SNM that is not disposed of on the day it is delivered to the facility.

64. "Disposal" is the locating of radioactive waste into a lift of the disposal embankment. Disposal does not include the storage of waste in containers on a lift when the container will ultimately be emptied, the

staging of containerized waste in the disposal embankment; or waste as "In Cell Bulk Disposal".

MANIFEST/SHIPPING REQUIREMENTS

65. The Licensee shall comply with UAC R313-15-1006 and UAC R313-25-33(8), Requirements for Low-Level Waste Transfer for Disposal at Land Disposal Facilities and Manifests.

66. The Licensee shall not accept radioactive waste for storage and disposal unless the Licensee has received from the shipper a completed manifest that complies with UAC R313-15-1006 and UAC R313-25-33(8).

67. The Licensee shall maintain copies of complete manifests or equivalent documentation required under Conditions 65 and 66 until the Executive Secretary authorizes their disposition.

68. The Licensee shall immediately notify the Executive Secretary or the Division's on-site representative of any waste shipment where there may be a possible violation of applicable rules or license conditions.

69. The Licensee shall require anyone who transfers radioactive waste to the facility to comply with the requirements in UAC R313-15-1006.

70. The Licensee shall acknowledge receipt of the waste within one (1) week of waste receipt by returning a signed copy of the manifest or equivalent document to the shipper. The shipper to be notified is the Licensee who last possessed the waste and transferred the waste to the Licensee. The returned copy of the manifest or equivalent documentation shall indicate any discrepancies between materials listed on the manifest and materials received.

71. The Licensee shall notify the shipper (e.g., the generator, the collector, or processor) and the Division when any shipment or part of a shipment has not arrived within 60 days after receiving the advance manifest.

72. The Licensee shall maintain a record for each shipment of waste disposed of at the site. At a minimum, the record shall include:

    A. The date of disposal of the waste;

    B. The location of the waste in the disposal site;

    C. The condition of the waste packages received;

    D. Any discrepancy between the waste listed on the shipment manifest or shipping papers and the waste received in the shipment;

    E. A description of any evidence of leaking or damaged packages or radiation or contamination in excess of applicable regulatory limits; and

F. A description of any repackaging of wastes in any shipment.

FINANCIAL ASSURANCE/CLOSURE

73. The Licensee shall maintain a Surety that satisfies the requirements of UAC R313-25-31 in an amount adequate to fund the decommissioning and reclamation of Licensees' grounds, equipment and facilities by an independent contractor. The Licensee shall annually review the amount of surety and submit a report of its findings to the Executive Secretary by August 31 each year. The Executive Secretary shall annually determine the required amount of surety under the Surety and shall require the Licensee to adjust the surety as necessary to reflect any increase in decommissioning and reclamation costs.

74. One (1) year prior to the anticipated closure of the site, the Licensee shall submit for review and approval by the Executive Secretary a site decontamination and decommissioning plan. As part of this plan, the Licensee shall demonstrate by measurements and/or modeling that concentrations of radioactive materials which may be released to the general environment, after site closure, will not result in an annual dose exceeding 25 millirems to the whole body, 75 millirems to the thyroid, and 25 millirems to any other organ of any member of the public.

75. In accordance with UAC R313-25-33(6), the Licensee shall submit a financial statement annually by March 31st of each year for the previous year.

SPECIAL HANDLING

76. Except while waste packages are being handled in the active areas of the Containerized Waste Facility, external gamma radiation levels shall not exceed 40 mR/hr at one meter from the surface of any emplaced waste package or from shielding placed around disposed waste containers. Measurements of radiation levels shall be taken one meter from exposed surfaces and other appropriate locations and recorded daily.

77. Reserved

78. The Licensee shall observe the following controls on waste handling at the Containerized Waste Facility:

A. Before unloading any waste container whose external gamma radiation at the surface exceeds 10 R/hr, an ALARA review shall be performed and documented and a pre-job briefing shall be conducted.

B. As part of the ALARA review, the Licensee shall determine and record (1) estimates of the radiation dose rates for the waste container, disposal unit working face, and any other potentially significant radiation sources; (2) expected durations of exposures to and distances from each radiation source; and (3) expected doses to each person involved in the actual disposal operation.

C. Before unloading any waste container whose external gamma radiation at the surface exceeds 50 R/hr, a practice run shall be conducted. The practice run shall involve shielding, container(s) filled with non-radioactive material, and handling equipment that are similar to those involved with the actual shipment. Similarity includes similar rigging and physical characteristics (e.g., weight, dimensions, and attachments). Those personnel who will participate in receiving, processing, handling, and disposing of the actual waste will participate in the practice run, using actual procedures.

D. On a case-by-case basis, the Executive Secretary may exempt the Licensee from conducting the required practice run, considering the results of earlier practice runs and actual experience handling waste containers with high radiation levels.

79. The Licensee shall notify the Division 24 hours in advance of conducting the practice runs described in Condition 78.

80. The Licensee shall notify in writing the Executive Secretary at the earliest possible date, but no later than 10 days before scheduled receipt of each shipment with contact radiation levels in excess of 200 R/hr. The notification shall include the anticipated dates of receipt and plan for disposal in the Containerized Waste Facility.

81. The Containerized Waste Facility Radiation Safety Manager or other qualified person he designates shall be present for and shall observe the receipt, processing, handling, and disposal of each waste package with contact radiation levels in excess of 200 R/hr.

82. The Licensee shall dispose of only closed containers in the Containerized Waste Facility. The Licensee shall not dispose of any breached waste container in the Containerized Waste Facility without first repairing the breached container or overpacking it in an undamaged container. The Licensee is authorized to open packages at its facility only to:

A. Repair or repackage breached containers.
B. Inspect for compliance with conditions of this license.
C. Confirm package contents.
D. Accomplish other purposes as approved by the Executive Secretary.

83. The Licensee shall handle and emplace LLRW packages in the Containerized Waste Facility such that packaging integrity is maintained during handling, emplacement, and subsequent backfilling. Waste packages deposited in the Containerized Waste Facility shall be protected from any adverse effects of operations which may damage them.

84. Except as specifically provided otherwise in this license, the Licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Utah Radiation Control Rules, Utah Administrative Code R313 shall govern unless the statements, representations, and procedures in the Licensee's application and correspondence are more restrictive than the rules.

A. License renewal application, revision 6, dated 16 March 1998.
B. Letter dated October 23, 1998.
C. Letter dated January 15, 1999.
D. Letters dated February 16, 1999, March 10, 1999, and March 23, 1999.
E. Letter dated April 19, 1999, and the U.S. Nuclear Regulatory Commission's Order dated May 7, 1999, and other administrative changes.
F. Letter dated July 15, 1999.
G. Letter dated September 1, 1999.
H. Letters dated July 15, 1999, June 28, 1999, August 27, 1999, October 19, 1998 and August 19, 1999.
I. Letters dated October 15, 1999, and November 4, 1999.
J. Letters dated June 3, 1999, November 5, 1999, February 16, 2000, and March 21, 2000.
K. Letters dated April 28, 2000, May 5, 2000, May 10, 2000, and June 6, 2000.
L. The following documents refer to the Class A disposal cell.
   (1) Letters dated September 24, 1999, March 6, 2000, April 14, 2000, July 21, 2000, July 26, 2000, August 8, 2000 and August 15, 2000.
   (2) Revised Run-On/Run Off Berm Calculations dated May 26, 2000.
   (3) Revised Engineering and Modeling Analysis dated June 19, 2000.
M. Request for License Amendment: Containerized Class A LLRW Disposal, dated Apr.12, 2001.
N. Engineering Justification Report, Addendum "Fifteen Percent Void Space Criteria" (Revision 1 dated October 10, 2001).
O. AMEC letter to Envirocare of Utah, Inc. "Placement of Drums and B-25 Containers with 15 Percent Voids; Envirocare Class A - Containerized Waste Facility Near Clive, Utah" (dated October 2, 2001).
P. AMEC letter to Envirocare of Utah, Inc. "Response to Interrogatory Number 2: Placement of HICs in Caissons; Envirocare Class A Disposal Facility Near Clive, Utah"(dated October 1, 2001).

UTAH RADIATION CONTROL BOARD

William J. Sinclair, Executive Secretary

Date